

# Modification in Cu-Zn Alloy Properties by 2 MeV Ni<sup>+</sup> Ions Irradiation

Shahbaz Ahmad<sup>1,2\*</sup>, Shazia Bashir<sup>2</sup>, Daniel Yousaf<sup>2</sup>, Mian Ahsan Ali<sup>2</sup>

<sup>1</sup>Department of Physics, University of Sargodha, Lahore Campus, Lahore, Pakistan

<sup>2</sup>Center for Advanced Studies in Physics, Government College University, Lahore, Pakistan

Email: \*shahbazcasp@gmail.com

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## Abstract

We investigate the effects of 2 MeV Ni<sup>+</sup> ion beam irradiation with various fluence ranging from  $15 \times 10^{11}$  to  $60 \times 10^{14}$  ions/cm<sup>2</sup> on the surface, structural and mechanical properties of Cu-Zn alloy. The modification in target properties after irradiation is confirmed by using various characterization techniques viz. SEM, XRD, UTM and Vickers micro-hardness tester. The SEM results illustrate the formation of nano sized craters with different diameters. Their average diameter decreases from 190 nm to 90 nm by increasing ion fluence. The XRD analysis of irradiated targets reveals that Ni<sup>+</sup> ion irradiation enhances the growth of (111) phase and its peak position varies due to ion induced tensile stresses in target matrix. Tensile and Vickers micro-hardness tests verify the mechanical properties of Cu-Zn alloy reduce monotonically upon irradiation. Various mechanisms such as generation, recombination, augmentation and annihilation of ion induced defects are responsible for this reduction. Understanding the relationships between various modified properties of irradiated target is essential for growing new advanced material by irradiation.

## Keywords

Ion Irradiation, Mechanical Properties, Ion Fluence, Frenkel Defects, Crystallite Size

## 1. Introduction

Ion irradiation is a valuable and innovative tool for advanced materials processing with novel surface, structural and mechanical properties e.g. for surface modification (texture, hardness, corrosion) and formation of metastable phases (ripples, surface patterning, plastic flow) [1]. During ion material interaction, development

of ion-induced vacancy and interstitial concentrations takes place due to elastic and inelastic collisions. Frenkel defects are generated from the collisions between incident high-energetic ions and host atoms of lattice systems [2]. These defects can be lost either through recombination of vacancies and interstitials or by reaction with a defect sink (void, dislocation, dislocation loop, grain boundary or precipitate). The formation, growth and dissolution of defect aggregates such as dislocation, dislocation loops, voids and other types of lattice defects depend on the diffusion of point defects and their reaction with the defect aggregates [3] [4]. The increase in diffusion or atom mobility enhancement in an irradiated surface is due to enhanced concentration of the defects and production of new defects. Consequently, the probability for their mutually recombination is much higher leading to a partial defect annealing during ion irradiation. These ion induced defects can modify the surface, structural and mechanical properties of irradiated target.

Cu alloy (Cu and Zn) is commonly utilized in various industries due to of its good formability, excellent strength to weight ratio, high corrosion resistance, appropriate hardness and ductility. Auto industries are always in need of strong and light automotive parts. Therefore, it is probable to manufacture several parts and products for automobile applications using the Cu-Zn alloy [5]. The properties of Cu-Zn alloy can be further modified by ion irradiation.

The modification of material properties of metals and their alloys by ion irradiation has been probed for several years. Zuo *et al.* [6] irradiated Ti-6Al-4V alloy by H<sub>2</sub> and N<sub>2</sub> ion beams. The results reveal that the mechanical properties (tensile and fatigue) of irradiated target reduce significantly after irradiation with H<sub>2</sub> and N<sub>2</sub> ions. Wang *et al.* [7] irradiated Zr-45Ti-5Al-3V alloy by 84 MeV C ions with various ion doses. The XRD results show no new phase is formed whereas position and intensity of diffraction peak change with C irradiation dose. Zelaya *et al.* [8] studied the effects of 300 KeV Cu<sup>+</sup> ions irradiation on properties of Cu-Zn-Al. The size, shape and density distribution of irradiation induced cavities as a function of different conditions of irradiation. Yu *et al.* [9] reported that the yield strength of the Cu nanowires decreases after ion irradiation by increasing irradiation energy. Hu *et al.* [10] studied the effect of Cl<sup>4+</sup> and C<sup>4+</sup> ions irradiation with 25 MeV energy on the mechanical properties of bulk metallic glass. The results revealed that the hardness of irradiated target decreased after Cl<sup>4+</sup> ion irradiation.

The objective of this research work to correlate the mechanical modification with surface and structural variation after ion irradiation. The surface morphological growth after ion irradiation is examined by Scanning Electron Microscope (SEM). X-Ray Diffractometer (XRD) is utilized to analyze the crystallographic structure of irradiated Cu-Zn alloy.

## 2. Experimental Details

The polycrystalline Cu-Zn alloy sheet (70 wt% Cu and 30 wt% Zn) with dimen-

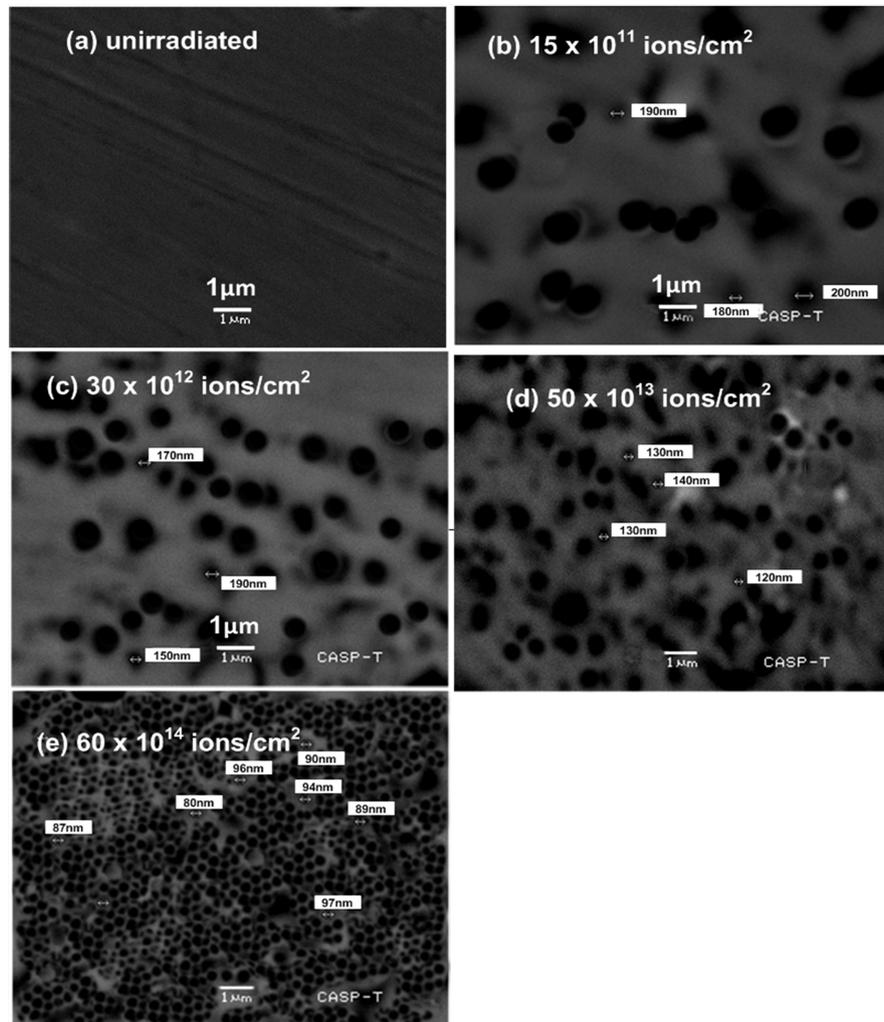
sions of  $15 \times 12 \times 6 \text{ Cm}^3$  was purchased from Alfa Aesar (United State of America). The yield and tensile strength of target at room temperature are 75 and 300 MPa respectively while its modulus of elasticity is 102 GPa. The five rectangular shaped targets of dimensions 60 mm length, 6 mm width and 3 mm thickness are cut from as-received Cu-Zn sheet with the help of computerized numerical control wire cutting machine. The target surfaces are abraded and polished by silicon carbide paper of different progressive grades ranging from 500 to 4000. After this these prepared targets are enclosed in pyrex glass tube evacuated up to a base pressure of  $10^{-6}$  Torr using a rotary pump followed by a diffusion pump. In order to relieve internal stresses and defects, these vacuum sealed pyrex glass tube containing targets are placed in a high temperature furnace (Nabertherm-LHT-02/18, Germany) and annealed at 774 K under vacuum condition ( $10^{-4}$  Torr). After annealing, these targets are ultrasonically cleaned by acetone and then mounted on target holder. Then four mounted targets are irradiated (one by one) with 2 MeV singly charged Ni ion beam for various ion fluences of  $15 \times 10^{11}$ ,  $30 \times 10^{12}$ ,  $50 \times 10^{13}$  and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at 300 K under ultra-high vacuum condition of  $\sim 10^{-9}$  Torr using Pelletron linear accelerator (6SDH-2 NEC USA). The Scanning Electron Microscope (SEM, JEOL-JSM-6480) technique is utilized to study the surface morphological changes before and after ion irradiation. The structural investigations are carried out using X-ray diffraction (XRD, X'Pert PRO-MPD) method.

### 3. Results and Discussion

#### 3.1. Surface Morphology

SEM micrographs of **Figure 1** depict (a) unirradiated and Ni ions irradiated targets for various Ni ion fluences of (b)  $15 \times 10^{11}$  ions/cm<sup>2</sup>, (c)  $30 \times 10^{12}$  ions/cm<sup>2</sup>, (d)  $50 \times 10^{13}$  ions/cm<sup>2</sup> and (e)  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

For the lowest fluence of  $15 \times 10^{11}$  ions/cm<sup>2</sup>, diffused and irregular shaped nanocraters with random density distribution are observed as revealed in **Figure 1(b)**. Increase in fluence of Ni ions from  $30 \times 10^{12}$  ions/cm<sup>2</sup> to  $60 \times 10^{14}$  ions/cm<sup>2</sup>, nano sized circular and elliptical shaped craters are observed in **Figures 1(c)-1(e)**. The density of nano craters increases by increasing the ion fluence, whereas their average diameter decreases from 190 nm to 90 nm. There is competitive phenomenon between thermal sputtering and redistribution by increasing fluence. With increase in ion fluence the probability of redistribution and refilling is larger than thermal sputtering, causing the reduced diameter of craters with increasing ion fluence. The surface morphology of Cu-Zn alloy is changed after ion induced melting, thermal and pressure spike, thermal sputtering, evaporation and deposition. These phenomena are responsible for the formation of nano meter sized craters [11]. The deposition of energy within the absorbing lattice of material increases by increasing the Ni ion fluence and is attributable to enhancement of accumulated and collective effects of ions. The ion



**Figure 1.** SEM images illustrating the surface morphology of (a) unirradiated and Ni ions irradiated targets for various ion fluences of (b)  $15 \times 10^{11}$  ions/cm<sup>2</sup>, (c)  $30 \times 10^{12}$  ions/cm<sup>2</sup>, (d)  $50 \times 10^{13}$  ions/cm<sup>2</sup> and (e)  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

induced stresses, shocks waves, elastic rebounds and ion induced stresses are produced in well-defined and localized regions and responsible for such kinds of features. When the energetic ions are incident on the target surface, they lead to sputtering of surface host atoms. Some incident ions are also scattered in different directions after collisions. If the energy of incident ion is greater than the binding energy of lattice atom then it breaks the chemical bonds between the host lattice atoms of the irradiated surface. The energy and momentum of incident ion are completely transferred to the host atom of the irradiated surface after elastic collision. After receiving this energy and momentum, the host atom are dislocated from their normal lattice position. Therefore Primary Knock on Atoms (PKAs) phenomenon is activated. These PKAs further share their momentum and energy with nearby host atoms which cause the generation of displacement spikes and thermal spike. The violent interaction is produced between host atoms in the central core of thermal spike [2]. Therefore, a drastic compress-

sion (up to GPa level) of the material atoms and significant increase of temperature up to a few thousand degrees is developed in the impact zone and subsurface. The induced pressure spike and heat spike lead to the material melting around the impact zone. The melted zone is produced close to the surface, where GPa pressure builds up due to the compression of material atoms which cannot be contained in surface and subsurface. Hence, the host atoms of irradiated surface can be pushed towards the surface by viscous flow. Consequently, surface craters are formed due to micro explosions [12]. The formation of surface crater is dependent on the intrinsic properties of target and energy of incident ions. More favorable conditions for the production of surface crater after ion irradiation are lower material density, low melting temperature of target material, low binding energy and smaller atomic displacements. The shape of the craters is dependent on the incident angle. Typically round in shape craters are formed at normal incidence. On the other hand when the angle is increased or decreased with respect to normal incidence it leads to produce alteration in shape of craters. At the oblique impact (60° angle) the shape of crater is usually elliptical [13]. Our targets were exposed at angle of 90° with respect to its surface. This angle was kept constant for irradiation. But a change in shape of nano craters is observed which is attributable to multiple scattering events. The increase in diameter of craters is attributed to coalescence process. The ion induced collision cascade develops into thermal spike regions where the hosts atoms of irradiated material are performed violent motion. Therefore the deposited energy of incident photons converts into heat energy. This heat energy cause to raise the surface temperature up to thousand degree Celsius. As a result molten zone is generated on the irradiated surface [14]. The ion induced shock liquefied and intense melting material refills the craters and basis reasons to reduce in their size [15]. The change in diameter of surface craters is due to ion induced imperfections in the lattice site of target surface such as vacancy, interstitial defects, small pits, dislocation loops and other heterogeneities. The non-uniform energy absorption, displacement spike, thermal spike, pressure spike and recrystallization are responsible for these imperfections [4].

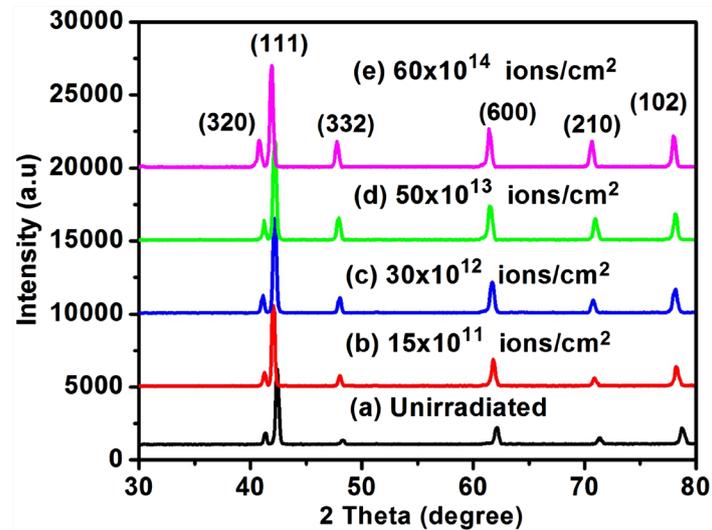
### 3.2. XRD Analysis

XRD spectra of unirradiated and Ni ion irradiated target sample are shown in **Figure 2**. The unirradiated target shows the existence of CuZn (320), CuZn (111), CuZn (332), CuZn (600), CuZn (210) and CuZn (102) planes reflection of Cu-Zn alloy at angles of 41.7°, 42.6°, 48.59°, 62.46°, 71.62° and 78.03° respectively [16].

The average crystallite size is evaluated by using Sherrer's formula [17].

$$\text{Crystallite Size}(D) = \frac{0.9\lambda}{\text{FWHM} \cos \theta} \quad (1)$$

where  $D$  is crystallite size,  $\lambda$  is the wavelength of X-rays (1.542 Å), FWHM is full width at half maximum, and  $\theta$  is the angle of diffraction.



**Figure 2.** XRD spectra of unirradiated (a) and Ni ions irradiated targets for various ion fluences of (b)  $15 \times 10^{11}$  ions/cm<sup>2</sup>, (c)  $30 \times 10^{12}$  ions/cm<sup>2</sup>, (d)  $50 \times 10^{13}$  ions/cm<sup>2</sup> and (e)  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

The length of dislocation lines in unit volume of the crystal material is called dislocation line density. The dislocation line density is calculated by following relation [17].

$$\text{Dislocation line density}(\delta) = \frac{1}{D^2} \quad (2)$$

The residual strain variations are evaluated by using following relation [17].

$$\text{Strain}(\varepsilon) = \frac{d - d_0}{d_0} \quad (3)$$

where  $\varepsilon$  is the induced strain,  $d$  is the observed and  $d_0$  is the standard plane spacing.

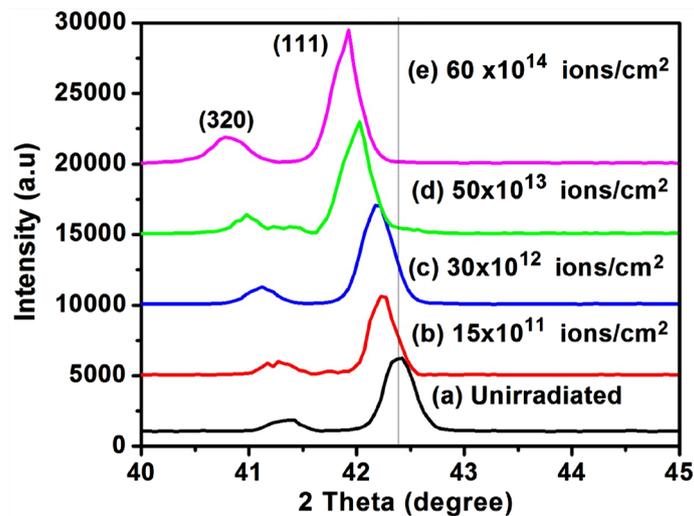
Induced stresses  $\sigma$  can be calculated by relation given below [17].

$$\text{Stress}(\sigma) = \varepsilon E \quad (4)$$

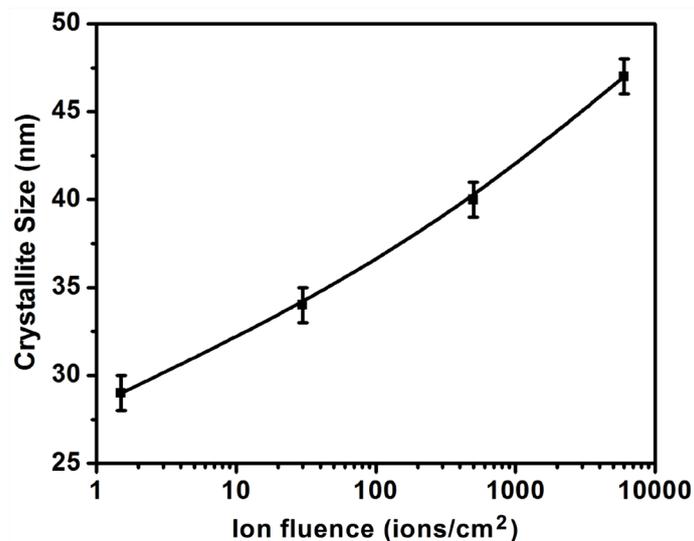
where  $\sigma$  is induced stresses,  $\varepsilon$  is the induced and  $E$  is the young's modulus, for Cu-Zn its value is 102 GPa [18].

It is found that the intensity of Cu-Zn (111) peak increases by increasing ion fluence (Figure 2). No new phase is formed after the ion irradiation. Wang *et al.* [7] also reported that no new phase is observed upon 84 MeV C ion irradiation. The close examination of Figure 3 and it is observed that the peak intensity of irradiated target surface for the plane of (111) increases monotonically by increasing ion fluence up to a maximum value of  $60 \times 10^{14}$  ions/cm<sup>2</sup> is attributed to reduction in density of generated defects by annihilation process after ion material interaction [19]. This improvement in peak intensity is due to the atomic diffusion of target matrix atom across the grain boundaries. Therefore, new alignment of host atom is formed after ion irradiation and this process is called crystal growth. The grains of the material consist of large number of crys-

tallites which have dissimilarity orientation. A crystallite may be composed of multiple domains. X-ray line profile analysis has been employed to indirectly find out the crystallite size, dislocation line density and ion induced stresses in lattice planes [20]. From **Figure 3** it is notable that diffraction peak of the plane (111) shifts to smaller angle which shows that lattice constant parameters (d-spacing and FWHM) are changed. Upon irradiation the inter-planar spacing (d-spacing) between the lattices is increased. As a result, the diffraction peak corresponding to (111) plane shifted towards lower value of angle [21]. After ion material interaction, **Figure 4** reveals graphically that a noticeable increment in crystallite size occurs from 29 nm to 47 nm with an increase in ion fluence from

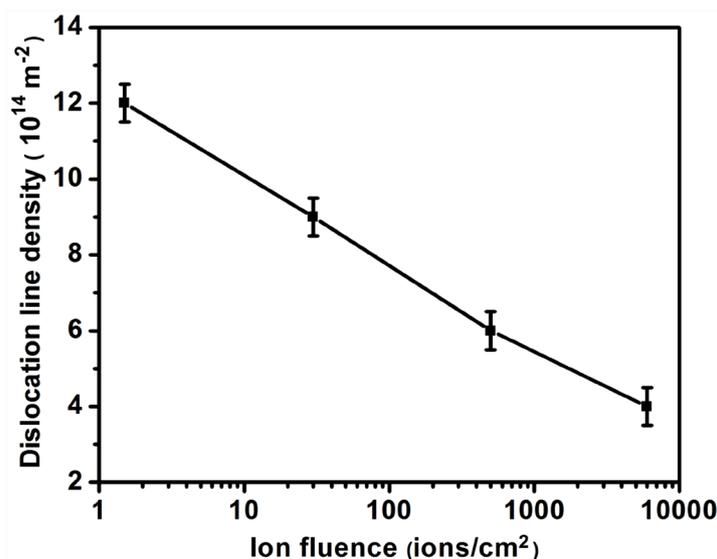


**Figure 3.** XRD spectra of unirradiated (a) and Ni ions irradiated targets for various ion fluences of (b)  $15 \times 10^{11}$  ions/cm<sup>2</sup>, (c)  $30 \times 10^{12}$  ions/cm<sup>2</sup>, (d)  $50 \times 10^{13}$  ions/cm<sup>2</sup> and (e)  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

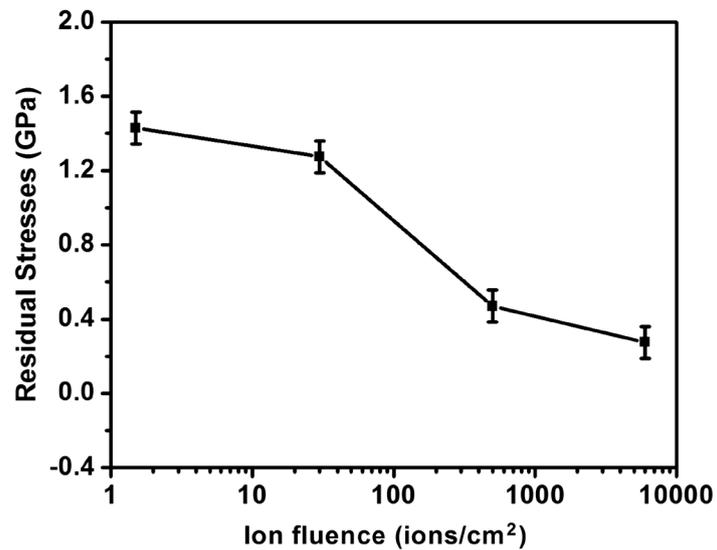


**Figure 4.** The variation of crystallite size of Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

$15 \times 10^{11}$  ions/cm<sup>2</sup> to maximum value of  $60 \times 10^{14}$  ions/cm<sup>2</sup>. The estimated crystallite size of unirradiated target is  $\sim 24$  nm. Ion irradiation and thermal annealing process lead to reduced strain field by more texturing. The probable justification for this kind of structure variation induced by thermal annealing and ion irradiation can be justified on the following assumption that the collective deposition of energy of total irradiated ions is more effective than single ion bombardment on the surface of target [22]. When the target surface is irradiated with Ni ions, the number of multiple collisions of incident ions with host atoms of target increases by increasing ion fluence. Therefore, the energy deposition by the bombarded ions on the target surface is usually appeared as thermal heating in the surface and subsurface of material. The extensive amount of ion energy is transferred to the material and is responsible for the development of very high temperature zone. Ion irradiation results in the form of local annealing of the material surface under the effect of transitory heating by electron and phonon coupling and is called thermal spikes [22]. This energy leads to minimize the strain field between the grains of irradiated target. Therefore, the small grains emerge into large grain. As a result further improvement occurs in crystallite quality of material upon irradiation. The ion irradiation activates the annihilation process of defects which is one of the major reasons for the enhancement in the crystallite size [23]. Liu *et al.* [1] reported that the average grain size increases by increasing ion dose. The effect of ion fluence on the dislocation line density and residual stress is illustrated in Figure 5 and Figure 6 respectively for predominant peak of (111) plane. Figure 6 depicts that the ion induced residual stresses in irradiated target material are tensile stresses in nature. With increasing Ni ion fluence, both the dislocation line density and residual stresses are reduced. This behavior can be explained on the base of two competing phenomena



**Figure 5.** The variation in dislocation density of Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

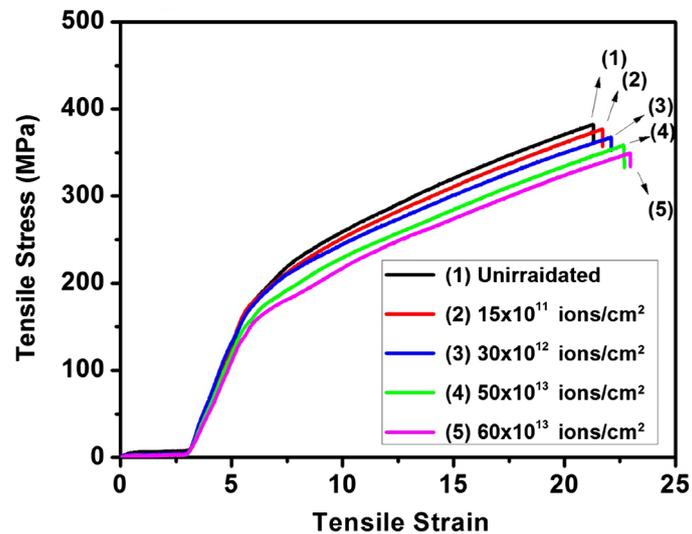


**Figure 6.** The variation of residual stress of Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

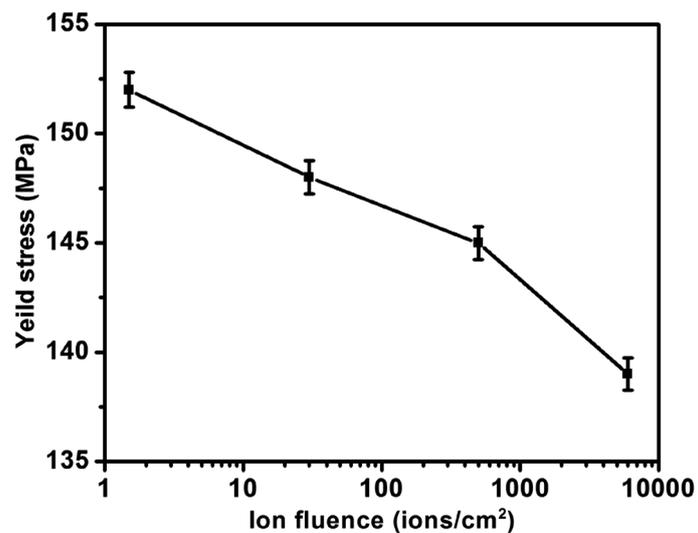
which are produced simultaneously during ion material interaction. The first phenomenon is production of vacancies and agglomeration of induced vacancies which are finally collapsed into dislocation loop. The other phenomenon is the transference of high amount of energy to the target surface via ion bombardment which leads to generation of high temperature zone on the surface and subsurface. As a result annihilation process is activated [24]. Although more vacancies are produced by increasing ion fluence but at the same time annihilation rate of these induced vacancies also increase. Therefore sink volume increases by increasing irradiation fluence. Because the surface damage by Ni ion irradiation is tremendously high and the surface recovery is impossible [25].

### 3.3. Tensile Testing

The stress strain curves of unirradiated and irradiated targets for various fluences are depicted in Figure 7. The mechanical properties of irradiated targets such as Yield Stress (YS) and Ultimate Tensile Strength (UTS) decreases monotonically by increasing ion fluence and is graphically represented in Figure 8 and Figure 9 respectively. The change in YS and UTS of target after ion material interaction is related with the improvement in crystalline size (Figure 4), a reduction in dislocation line density (Figure 5) and ion induced thermal annealing. Similar results have been reported by Zuo [6], in which after H and N ions irradiation the yield and tensile strength of Ti-6Al-4V alloy are significantly decreased than that of the as-received target sample. The reduction in YS and UTS is also explainable on the basis of thermal spike which is produced by cascade collisions after ion irradiation. The main cause of radiation damage are elastic interactions between incident ions and host atoms of irradiated target lattice, as a result these host atoms displace from their normal stable lattice position.

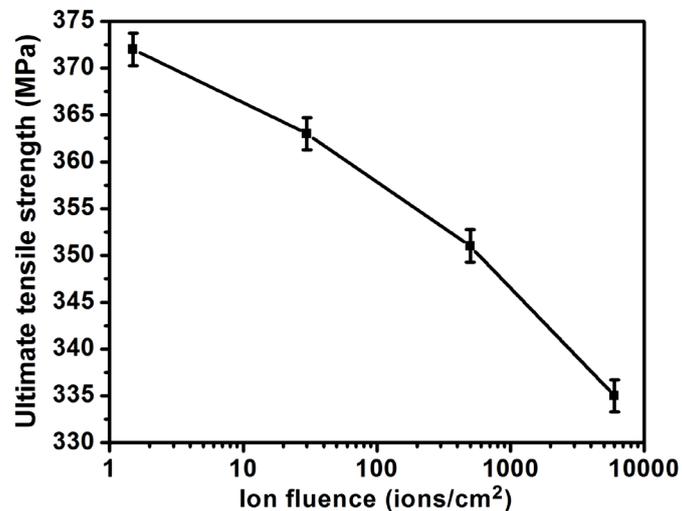


**Figure 7.** The comparison of stress-strain curves measured by tensile testing machine of unirradiated and Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.



**Figure 8.** The variation of Yield Stress (YS) of Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

When the value of transferred energy ( $E_m$ ) to the host atoms due to elastic collision is greater than displacement energy ( $E_d$ ) then the atomic displacement produces. The displacement energy is intrinsic property of the material, the range of its value 20 - 40 eV [26] [27]. But Broeders and Konobev [27] described that the displacement energy value can change in wider range from 10 eV to 600 eV. The maximum energy transferred to the host atom is directly proportional to ion fluence as well as Ni ion energy and inversely proportional to the atomic mass of the irradiated target material. In this regards, by increasing number of impacts and consequential radiation damage deeply depends on the

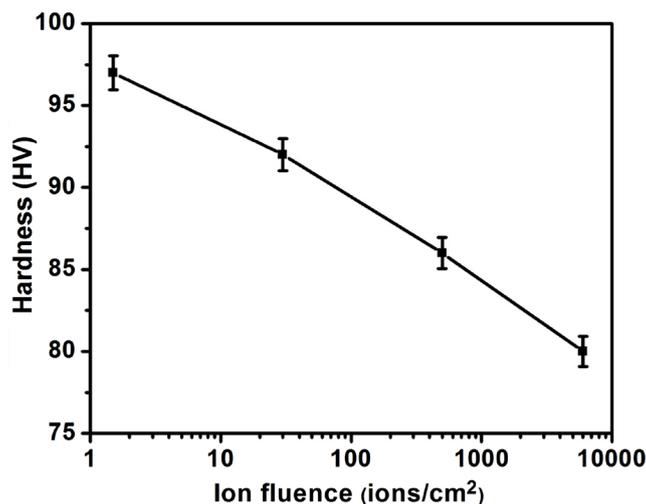


**Figure 9.** The variation of Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

target lattice and cohesive forces of material. The fast moving knock on atoms (PKA) is produced by Ni ions host atom elastic interaction. The energy of PKA atoms ( $E_{PKA}$ ) can be easily defined by the difference between  $E_d$  and  $E_m$ . The displaced PKA will collide with other host atoms elastically and produce displacement spike. There is production of thermal spike in the displacement spike just after the ballistic phase. Where the energy of the PKAs is distributed among many atoms to the central core of disorder. As a result temperature of surface and subsurface is increased. The induced temperature during collision cascade is higher than melting temperature of irradiated material for at least  $10^{-12}$  second. Therefore self and ion induced vacancies and interstitial atoms have higher mobility which leads to the thermal annihilation of vacancies and interstitial defects. The average number of Frenkel pair (vacancies and interstitial defects) decrease with increase in the size of thermal spikes [28].

### 3.4. Micro-Hardness

**Figure 10** reveals decreasing trend in micro-hardness with increasing Ni ion fluence from  $15 \times 10^{11}$  ions/cm<sup>2</sup> to  $60 \times 10^{14}$  ions/cm<sup>2</sup>. Due to ion induced thermal spike, energy spike and displacement spike, lattice distortion is produced after irradiation. This causes production of Frenkel pairs defects and their annihilation in the normal lattice site of crystal. Therefore decrease in micro-hardness of irradiated target is observed. The reduction in hardness after ion irradiation is directly related to crystallite size and dislocation line density as shown in **Figure 4** and **Figure 5** respectively. The increase in crystallite size is main reason to decrease the hardness. During ion metal interactions, the energy absorbed is used to increase the mobility of existing and ion induced dislocation defects. As a result the rate of annihilation is dominant than generation of vacancies and interstitial defects [29]. Another vital reason for the reduction in



**Figure 10.** The variation in the microhardness of Ni ions irradiated target for various ion fluences of  $15 \times 10^{11}$  ions/cm<sup>2</sup>,  $30 \times 10^{12}$  ions/cm<sup>2</sup>,  $50 \times 10^{13}$  ions/cm<sup>2</sup> and  $60 \times 10^{14}$  ions/cm<sup>2</sup> at constant energy of 2 MeV.

hardness of target surface after ion solid interaction is ion induced thermal sputtering [30].

#### 4. Conclusions

The 2 MeV Ni<sup>+</sup> irradiation effects on the surface structural and mechanical properties of Cu-Zn alloy have been investigated successively. The target surface is irradiated with fluence range of  $15 \times 10^{11}$  to  $60 \times 10^{14}$  ions/cm<sup>2</sup> by using Pelletron linear accelerator. SEM analysis reveals the formation of nano sized craters after ion irradiation. The density of nano craters increases by increasing the ion fluence, whereas their average diameter decreases from 190 nm to 90 nm.

The XRD analysis of irradiated targets no new phase is formed while growth improvement of (111) phase is clearly observed because of ion irradiation. The development in crystallite size and reduction in dislocation line density are observed by increasing irradiation fluence. It is attributed to the fact that during ion irradiation, vacancies and interstitials defects generated and intrinsic and extrinsic defects turned into collapses during self-annealing phenomenon. This ion induced self-annealing mechanism is responsible for diffraction peak shifting towards the lower angle. The change in YS, UTS and micro-hardness of target after ion material interaction is related with the improvement in crystalline size, a reduction in dislocation line density and ion induced thermal annealing. Due to ion induced thermal spike, energy spike and displacement spike, lattice distortion is produced after irradiation. This causes production of Frenkelpairs defects and their mutual annihilation in the normal lattice site of crystal.

#### Novelty Statement

In this paper, the modification in properties of brass after 2 MeV Ni<sup>+</sup> ions irradiation has been explored. The surface modification has been correlated with the

structural as well as mechanical behavior of irradiated brass for the first time. The variation in hardness as well as the strength of material is observed with increasing ion fluence.

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